Ecole de Métallurgie Fondamentale

Action Nationale de Formation (ANF)

22-25 Octobre 2012, Aussois

Métaux et Alliages sous Irradiation

P. Pareige Groupe de Physique des Matériaux UMR CNRS 6634, Université et INSA de Rouen

A. Barbu, B. Radiguet SRMP, CEA Saclay CIEMAT, Espagne EDF, Les Renardières





Fig. 1.12. Failure by brittle fracture of thick-walled cylindrical pressure vessel during hydraulic test (By courtesy of John Thompson Ltd., Wolverhampton)



Outline

Interaction ion-matter

Ballistic Damage (primary damage)

- Frenkel pairs
- Atomic Displacement Cascade
- Disorder induced by ballistic damage

Properties of point defects and their clusters :

- Structure
- Mobility

Slow evolution (secondary damage) :

- Evolution of the point defects population:
- Consequence of the super-saturation of point defects
 - Agglomeration of point defects
 - Enhanced phase transformation
 - Induced Segregation and precipitation

Macroscopic Consequences

Outline

Interaction ion-matter

Ballistic Damage (primary damage)

- -Frenkel pairs
- -Atomic Displacement Cascade
- -Disorder induced by ballistic damage

Properties of point defects and their clusters :

- -Structure
- -Mobility

Slow evolution (secondary damage) :

- -Evolution of the point defects population:
- -Consequence of the super-saturation of point defects
 - Agglomeration of point defects
 - Enhanced phase transformation
 - Induced Segregation and precipitation

Macroscopic Consequences

Origine du dommage d'irradiation: collision élastique

Neutron -> Chocs élastiques avec les noyaux



Collisions Elastique et Inélastique

Stopping and Ranging of Ions in Matter (SRIM)

+ 1.25 un

10-2

0

2

4





10 12

8

6 Distance into solid (u)

Dommage ballistique : seuil de déplacement

 E_d = displacement threshold

 $1/ T < E_{d}$: Phonon – Recombination

 $2/T > E_d$ but is low : single Frenkel pair

(one vacancy + one interstitial)



Dommage ballistique : seuil de déplacement

-



Metal	Lattice (c/a)	$E_{d,\min}$ (eV)	$E_{\rm d}~({\rm eV})$	
Al	fcc	16	25	
Ti	hcp (1.59)	19	30	
V	bcc	_	40	
Cr	bcc	28	40	
Mn	bcc	_	40	
Fe	bcc	20	40	
Co	fcc	22	40	
Ni	fcc	23	40	
Cu	fcc	19	30	
Zr	hcp	21	40	
Nb	bcc	36	60	
Mo	bcc	33	60	
Та	bee	34	90	
W	bcc	40	90	
Pb	fcc	14	25	
Stainless steel	fcc		40	

3/ If T>>E_d Atomic displacement cascade

Modèle de Kinchin-Pease

1- Cascade = suite de collisions à 2 corps (sphères dures)

2- $P_d(T) = 1$ si $T > E_d$ et 0 sinon

3- Pas d'énergie transférée au réseau ou sous forme d'excitation électronique

4- si T > E_c , pas de défauts supplémentaires

5- Arrangement aléatoire des atomes



.... As far as $E > E_d$





Modèle de Norgett, Robinson, Torrens (NRT)

- Potentiel d'interaction plus réaliste
- Excitation électronique

$$N_{\rm d} = \frac{\kappa E_{\rm D}}{2E_{\rm d}} = \frac{\kappa (T - \eta)}{2E_{\rm d}}$$

→ Efficacité de déplacement (κ):



Recoil energy (eV)

- → Energie de dommage (E_D):
 - T : énergie total du PKA η: énergie perdue par excitation électronique dans la cascade



NRT

Autre moyen d'étudier la cascade de déplacement : la modélisation : dynamique moléculaire



Sim. Dynamique Moléculaire: CEA Saclay

$$\frac{d^2 \, \vec{r}_i}{d \, t^2} = m_i^{-1} \left(- \frac{\partial E}{\partial \vec{r}_i} \right)$$



[R. E. Stoller, JNM, 1997]

Formation de sous cascades à haute énergie



4 keV Displacement cascade in Iron Molecular Dynamic Simulation (C. Domain - EDF)

Qu'est ce qu'un résidu de cascade....?

Cascade in Fe, 10 ps after the collision Pka of 20 keV.

Molecular Dynamic Simulation

Remains after 10 ps

- isolated Interstitials.
- isolated vacancies.
- Clusters of interstitials.
- Clusters of vacancies.



Défauts ponctuels et amas de défauts ponctuels dans les résidus

distribution of interstitial and vacancy clusters in cascades induced by primary iron ions (1 to 30 keV) in Fe (Vascon 1997)



A large portion of point defects is in the form of clusters

dpa DM versus dpa NRT



Le nombre de dpa donné par la dynamique moléculaire (DM) nettement inférieur aux dpa NRT

Quelques ordres de grandeur des taux de dommage

	Flux neutrons m ⁻² s ⁻¹ (n>1MeV)	dpa _{NRT} /s	dpa _{NRT} fin de vie	T °C	Pb associés	
REP éléments de coeur	10 ¹⁷	3 10 ⁻⁸	qqs dpa	≤ 400	Internes : corrosion sous contrainte assistée par l'irradiation gaines : fluage, croissance, corrosion	
REP Cuve	10 ¹⁵	3 10 ⁻¹⁰	0.2	290	Fragilisation	
RNR gaines	3 10 ¹⁸	10 ⁻⁶	150	550	Gonflement	

Réacteurs

Particules chargées

Accélérateur d'ions	10 ⁻³ - 10 ⁻⁵ dpa _{NRT} /s
Microscope électronique à haute tension (électrons)	10 ⁻³ - 10 ⁻⁵ dpa _{NRT} /s
Accélérateur d'électrons	10 ⁻⁹ dpa _{NRT} /s



Fig. 3.7. Difference in damage morphology, displacement efficiency and average recoil energy for 1 MeV particles of different type incident on nickel (after [6])

Effet du dommage ballistique: Désordre chimique

Disorder

May be kept and observed at low temperature







Disordered areas induced by displacement cascades in an ordered Cu₃Au alloy Lost of the crystalline structure in the cascade, for some compounds

Amorphisation under irradiation of $Zr(Fe,Cr)_2$ precipitate in Zircaloy (Fe back to solution).



Approximation

an alloy with clustering tendency will achieve under irradiation the macro state it would have outside irradiation at a higher effective temperature $\rm T_{\rm eff}$

$$T_{eff} = T \left(1 + \frac{D^{bal}}{D_{ch}} \right)$$

In alloy with ordering tendency, owing to the flux and temperature : enhancement of the ordering kinetic of a disordered solid solution (quenched) or disordering of an ordered alloy.

Modification of the phase diagram



Wilkes 1979

Outline

Interaction ion-matter

Ballistic Damage (primary damage)

- -Frenkel pairs
- -Atomic Displacement Cascade
- -Disorder induced by ballistic damage

Properties of point defects and their clusters :

- -Structure
- -Mobility

Slow evolution (secondary damage) :

- -Evolution of the point defects population:
- Consequence of the super-saturation of point defects
 - Agglomeration of point defects
 - Enhanced phase transformation
 - Induced Segregation and precipitation

Macroscopic Consequences

Structure des défauts ponctuels

Interstitial

In compact structures (metals), for Self Interstitial Atoms (SIA), the stable configuration is N atoms sharing the same crystallographic site (dumbbell or crowdion) configuration.



Vacancy

= empty crystallographic site



Energy of formation (\mathbf{E}_{v}^{f}) : 0.6 to 2 eV Relaxation Volume (Vrv):

weak (-0.2 Ω)

Point defects and small clusters

Di-Interstitials





Di- and tri-vacancies



SIA dans Fe- α

Configurations













a) <100> dumbbell

- b) <110> dumbbell
- c) <111> dumbbell
- d) <111> crowdion
- e) interstitiel octaédrique
- f) interstitiel tétraédrique

SIA dans Fe- α

Energie de formation des différentes configurations



Configuration la plus stable <110> dumbbell

SIA dans Fe- α

Mécanismes de migration

- Mécanisme de rotation/translation (DM)
 - Rotation <110> \Rightarrow crowdion

 $(0,13 < E_m < 0,25 \text{ eV})$

- Translation <111> ($\approx 0.04 \text{ eV}$)
- Mécanisme de Johnson (Ab-initio)



Configuration

• <u>n < 5</u> :

- ab-initio : ensemble de dumbbells <110> parallèles



Tétra-interstitiel <110> dumbbells

E_f = 11,05 eV

- DM : ensemble de crowdions <111> parallèles



Tétra-interstitiel <111> crowdions

 $E_f = 14,4 \text{ eV}$ (Wirth avec potentiel d'Ackland)

Configuration

•<u>n≥5</u>:

- ab-initio et DM : ensemble de crowdions <111> parallèles



Hepta-interstitiel <111> crowdions <u>Ab-initio</u>: $E_f = 16,6 \text{ eV}$ <u>DM</u> : $E_f = 21,1 \text{ eV}$ (Soneda avec potentiel de Johnson)

Travaux récents : C15 Laves structure



Irradiation-Induced Formation of Nanocrystallites with C15 Laves Phase Structure in bcc Iron

M.-C. Marinica, F. Willaime,* and J.-P. Crocombette

CEA, DEN, Service de Recherches de Métallurgie Physique, F-91191 Gif-sur-Yvette, France (Received 28 September 2011; revised manuscript received 22 November 2011; published 11 January 2012)

A three-dimensional periodic structure is proposed for self-interstitial clusters in body-centered-cubic metals, as opposed to the conventional two-dimensional loop morphology. The underlying crystal structure corresponds to the C15 Laves phase. Using density functional theory and interatomic potential calculations, we demonstrate that in α -iron these C15 aggregates are highly stable and immobile and that they exhibit large antiferromagnetic moments. They form directly in displacement cascades, and they can grow by capturing self-interstitials. They thus constitute an important new element to account for when predicting the microstructural evolution of iron base materials under irradiation.

DOI: 10.1103/PhysRevLett.108.025501

PACS numbers: 61.72.Bb, 61.72.jj, 61.80.Az, 61.82.Bg



FIG. 1 (color online). Structure of small C15 interstitial clusters in the bcc lattice. (a)–(c) Representation by vacancies (blue cubes) and interstitials (orange spheres) of the di-, tri-, and tetra-interstitial clusters. For the di-interstitial, the atoms of the bcc lattice at the center and at the edges of the Z16 Frank-Kasper polyhedron are also represented (green spheres). (d),(e) Skeleton representation, i.e., without the vacancies and the cubic lattice and with the atoms at the center of the Z16 polyhedra in green, of the hexa- and octa-interstitials. (f) Unit cell of the MgCu₂ C15 Laves structure, with the Mg atoms in green and the Cu atoms in orange. For every cluster with 3 SIAs or more, the variant with the lowest energy found within DFT in Fe is represented.

Amas d'interstitiels dans Fe- $\!\alpha$

Migration





- migration dans la direction <111> ($E_m < 0.1eV$)

- changement de direction par activation thermique

 $(E_{a2l}=0.09eV \text{ et } E_{a3l}=0.27eV)$

migration 1D/3D

Trajectoire d'un tri-interstitiel dans du fer- α à 1000K

• n ≥ 4 :

glissement 1D dans la direction <111>

→ migration 1D



CEA Saclay



Amas d'interstitiels dans Fe- α

Migration

		E _m (eV)	D ₀ (cm ² .s ⁻¹)		
Marian		$0,059 + 0,067 / n^{1,3}$	8,98.10 ⁻³ n ^{-0,61}		
Soneda		0,061	4,6.10-4		
Bacon		0,022-0,026	_		
Wirth	n=19	0,023	1,7.10-3		
	n=37	0,052	6,5.10-3		
Osestky		0,02-0,05	-		
Kuramoto n=200		0,2	-		



n=20 17cm < x < 68cm pendant 14 jours à 20°C



n=200 0,95cm < x < 33cm pendant 14 jours à 20°C

Outline

Interaction ion-matter

Ballistic Damage (primary damage)

- -Frenkel pairs
- -Atomic Displacement Cascade
- -Disorder induced by ballistic damage

Properties of point defects and their clusters :

- -Structure
- -Mobility

Slow evolution (secondary damage) :

- -Evolution of the point defects population:
- -Consequence of the super-saturation of point defects
 - Agglomeration of point defects
 - Enhanced phase transformation
 - Induced Segregation and precipitation

Macroscopic Consequences

Evolution de la population de défauts ponctuels Mécanismes fondamentaux

Recombination

 $\Box + \overset{\bigcirc}{} = \bigcirc$ (annihilation)

<u>Recombination rate</u>: $K_{iv} = 4\pi r_{iv} (D_i + D_v) \sim 4\pi r_{iv} D_i = R D_i$

• Elimination on sinks

- grain boundaries,
- free surfaces,
- dislocation lines,
- interfaces ...

$$\frac{\text{interstitials}: i + s = s}{K_{is} = 4\pi r_{is} D_{i}}$$

$$\frac{\text{vacancies}: v + s = s}{K_{vs} = 4\pi r_{vs} D_{v}}$$

Classical mean field approach (cinétique chimique homogène, rate equation, etc.)



/ Isotrop system for the diffusion as in cubic structures/ Constant concentration of sinks

/ Vacancies equilibrium concentration neglected

The homogeneous system is characterised by absorption coefficients where index j characterise the nature of the absorbing object

Equation bilan



or (equivalent to)

$$\frac{dC_{v}}{dt} = G - R D_{i}C_{v}C_{i} - K_{v}D_{v}C_{v}$$
$$\frac{dC_{i}}{dt} = G - R D_{i}C_{v}C_{i} - K_{i}D_{i}C_{i}$$

where
$$\begin{aligned} K_{vs} \ C_s &= K_v \ D_v \\ K_{is} \ C_s &= K_i \ D_i \end{aligned}$$

with
$$K_{\theta} = \sum_{s} k_{\theta s}^{2}$$

 $k_{\theta s}^{2} = strength of sink s on defect $\theta$$

Elimination on sink

The rate of elimination of point defects (DP) of α type on sink j is written when diffusion is the controlling process (not reaction at interface)



Damask, Wiedersich, Brailsford, Bullough, Nichols

Expression of coefficients

Z is the bias factor (simple way to take into account the interaction)

sink		Q			$Z_{n,\alpha}$	
cavity			$4\pi r_c N_c Z_{c,\alpha}$	$Z_{c,i} = Z_{c,v} = 1$		=1
Dislocatio	on loop		$4\pi r_b N_b Z_{b,\alpha}$	$Z_{b,\alpha} = \frac{\pi}{\ln(8r_b / r_{p,\alpha})}$		$\frac{r}{r_{p,\alpha}}$
disloca	ation		$ ho_{d}Z_{d,lpha}$	$Z_{d,\alpha} = \frac{2\pi}{\ln\left\{1/\left[r_{p,\alpha}(\pi\rho_d)^{1/2}\right]\right\}}$		$\frac{\pi}{(\pi\rho_d)^{1/2}}$
Grain bo	undary		$\frac{57.6}{d^2}$	$Z_{gb,lpha}$	$=k_{\alpha}^{\dagger}d$	/9.6

Z=1 if no interaction taken into account

Also, for Dislocations and dislocation loops : $\frac{Z_d^i}{Z_d^i} > \frac{Z_d^v}{Z_d^i}$

there is a stronger interaction for interstitial than vacancy because r_p is ~ $\frac{V_i^r > V_v^r}{V_i}$



commonly used:
$$\frac{Z_d^i \approx 1.1; 1.2}{Z_d^v \approx 1}$$

<u>Cavities, grain boundaries and surfaces</u> are neutral sinks (at first order): $Z^{i} = Z^{v} = 1$

Low temperatures and low density of sinks



Low temperatures and intermediate density of sinks



 $G = K_0$

Low temperatures and high density of sinks



Livre G. Was

High temperatures



Examen...



Calculez τ_3 dans le cas d'un réacteur à eau pressurisée Hypothèse: on suppose que les puits sont majoritairement des dislocations avec ρ =10⁹cm⁻².

Correction...

$$K_v^{\alpha} = \rho Z_v^{\alpha} = \rho$$

$$\tau_3 = 1/(3.510^{-12} * 10^9) = 285s$$

Court en comparaison de la durée de vie...



Point defect concentration versus temperature



Three domains of temperature may be define:



Défauts étendus (amas de défauts ponctuels)

Agglomeration → defect clusters





Extended Point defects Clusters



Evolution of point defects clusters

Classical mean field approach





General equations (hypo: only monomers are mobile)

$$\frac{monomères}{dC_{1\theta}} = G_{1\theta} + \beta_{2\theta}^{1\theta}C_{2\theta}C_{1\theta'} + 4\alpha_{2\theta}^{1\theta}C_{2\theta} + \sum_{n=3}^{n=3} (\alpha_{n\theta}^{1\theta}C_{n\theta}) - \sum_{n=2}^{n=2} (\beta_{n\theta}^{1\theta}C_{n\theta} + \beta_{n\theta}^{1\theta}C_{n\theta'})C_{1\theta} - 4\beta_{1\theta}^{1\theta}C_{1\theta}C_{1\theta} - R_{iv}C_{1\theta}C_{1\theta'} - K_{1\theta}^{d}(C_{1\theta} - C_{1\theta}^{eq}) - K_{i\theta}^{sk}(C_{1\theta} - C_{1\theta}^{eq}) - K_{i\theta}^{sk}(C_{1\theta} - C_{1\theta}^{eq})$$

$$\frac{dimères}{dt} = G_{2\theta} + 2\beta_{i\theta}^{1\theta}C_{1\theta}C_{1\theta} + \beta_{3\theta}^{1\theta'}C_{1\theta}C_{3\theta} + \alpha_{3\theta}^{1\theta}C_{3\theta} - 2\alpha_{2\theta}^{2\theta}C_{2\theta} - \beta_{2\theta}^{1\theta}C_{1\theta}C_{2\theta} - \beta_{2\theta}^{1\theta'}C_{1\theta}C_{2\theta}$$

$$\frac{dC_{n\theta}}{dt} = G_{n\theta} + \beta_{(n-1)\theta}^{1\theta}C_{1\theta}C_{(n-1)\theta} + (\beta_{(n+1)\theta}^{1\theta'}C_{1\theta'} + \alpha_{(n+1)\theta}^{1\theta})C_{(n+1)\theta} - (\alpha_{n\theta}^{1\theta} + \beta_{n\theta}^{1\theta}C_{1\theta} + \beta_{n\theta}^{1\theta'}C_{1\theta'})C_{n\theta}$$

Diffusion of in pure metal

Theory of diffusion \rightarrow

 $D^* = \alpha_v D_v C_v + \alpha_i D_i C_i$

 α : correlation factor

- Excepted at high T, $C_v >> C_v^{eq}$
- Diffusion via interstitials only occurs under irradiation ($C_i^{eq} \sim 0$)





Irradiation enhanced precipitation

If cross coefficient can be neglected, the only effect of irradiation is to sustain a large defect supersaturation which:

- → Increase the solute diffusion coefficient
- → must not modify the thermodynamic of the system







 $V = 45 \times 45 \times 100 \text{ nm}^3$

Parameters influencing D*



- Dislocation density (concentration of sinks)
- Production rate
- Sink strength

Solute transport under irradiation in AB alloy

Mécanismes lacunaire

(avec interaction B-v positive)





→ modification de la concentration locale au niveau du puits de défauts $\frac{\partial c_B}{\partial t} = -div \mathbf{J}_B$

Mécanisme interstitiel

Solute transport under irradiation in AB alloy

Annihilation of point defects at sinks -> Gradient of concentration



→ Flux of point defects toward sinks

By coupling, point defect fluxes result in a flux of solute atoms (B)

$$\begin{cases} \mathbf{J}_{v} = -(D_{v} \nabla c_{v} + d_{Bv} c_{v} \nabla c_{B}) \\ \mathbf{J}_{i} = -(D_{i} \nabla c_{i} + d_{Bi} c_{i} \nabla c_{B}) \\ \mathbf{J}_{B} = -(D_{B} \nabla c_{B} + c_{B} (d_{Bv} \nabla c_{v} + d_{Bi} \nabla c_{i})) \end{cases}$$

Zones dénudées au voisinage des joints de grains

- Les zones dénudées en défauts ponctuels se manifestent par l'existence de zones dénudées en boucles.
- Elles n'apparaissent pas pour tous les joints (L'efficacité des joints de grains pour l'élimination des défauts ponctuels dépend de leur nature).



Alliage à base de Nickel irradié avec des électrons de 2 MeV (5 10⁻⁹ dpa/s, 300°C, 8. 10⁻⁵dpa).

FeNiCrSi model alloy : 5 MeV Ni++



Outline

Interaction ion-matter

Ballistic Damage (primary damage)

- -Frenkel pairs
- -Atomic Displacement Cascade
- -Disorder induced by ballistic damage

Properties of point defects and their clusters :

- -Structure
- -Mobility

Slow evolution (secondary damage) :

- -Evolution of the point defects population:
- Consequence of the super-saturation of point defects
 - Agglomeration of point defects
 - Enhanced phase transformation
 - Induced Segregation and precipitation

Macroscopic Consequences



Dimensional variations



Swelling

Only happen when the vacancy flux towards cavities is more important than the interstitial one



500°C irradiation with Cu ions of 500 keV at 3 10⁻³ dpa/s; Oxygen 30, 70 et 110 ppm (Glowinsky 1976)



Figure I-20

Gonflement macroscopique de ~33% en volume, et ~11% en linéaire, observé dans un acier de type 316 CW, irradié à 510°C, à 75 dpa dans le RNR EBR-II. L'échantillon à gauche est non irradié et celui de droite est irradié [77].

Hardening

- Increase of the tensile strength
- Decrease of the elongation.



Evolution des courbes de traction d'un acier austénitique (316) irradié avec des neutrons, à 325°C, en fonction de la fluence en dpa.



Fig. 15.3. Schematic illustration of mechanistic issues believed to influence crack advance during IASCC of austenitic stainless steels in LWRs (from [6])

EVOLUTION DE LA MICROSTRUCTURE DES METAUX ET ALLIAGES METALLIQUES SOUS IRRADIATION



